

October 1, 2004

Mr. James Scarola, Vice President
Shearon Harris Nuclear Power Plant
Carolina Power & Light Company
Post Office Box 165, Mail Code: Zone 1
New Hill, North Carolina 27562-0165

SUBJECT: SHEARON HARRIS NUCLEAR POWER PLANT, UNIT 1 - REQUEST FOR
ADDITIONAL INFORMATION REGARDING NRC BULLETIN 2004-01,
"INSPECTION OF ALLOY 82/182/600 MATERIALS USED IN THE FABRICATION
OF PRESSURIZER PENETRATIONS AND STEAM SPACE PIPING
CONNECTIONS AT PRESSURIZED-WATER REACTORS"

Dear Mr. Scarola:

The Nuclear Regulatory Commission (NRC) staff has reviewed and evaluated your response dated July 27, 2004, to NRC Bulletin 2004-01, "60-day Response to NRC Bulletin 2004-01 for the Inspection of Alloy 82/182/600 Materials Used in the Fabrication of Pressurizer Penetrations and Steam Space Piping Connections at Pressurized-Water Reactors," for the Shearon Harris Nuclear Power Plant, Unit 1 (HNP). We have determined that more information is needed to complete our review. Our Request for Additional Information (RAI) for HNP is enclosed. We request that you to respond to the RAI within 30 days of receipt of this letter. If you have any questions, please contact me at 301-415-3025.

Sincerely,

/RA/

Chandu P. Patel, Project Manager, Section 2
Project Directorate II
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-400

Enclosure: RAI

cc w/enclosure:
See next page

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REQUEST FOR ADDITIONAL INFORMATION
FOR NRC BULLETIN 2004-01 RESPONSE
Shearon Harris Nuclear Power Plant, Unit 1
Carolina Power & Light Company
Docket No. 50-400

The NRC staff has completed its review of your response dated July 27, 2004, to NRC Bulletin 2004-01, "60-day Response to NRC Bulletin 2004-01 for the Inspection of Alloy 82/182/600 Materials Used in the Fabrication of Pressurizer Penetrations and Steam Space Piping Connections at Pressurized-Water Reactors." Based on the NRC staff's review, please provide a supplemental response that addresses the following question.

Your response to Bulletin 2004-01 Question (1)(c) did not clearly communicate your intentions with respect to ensuring that an appropriate dialogue would be established with NRC technical staff in the event that circumferential primary water stress corrosion cracking (PWSCC) is identified at any locations covered under the scope of Bulletin 2004-01. The NRC staff addressed this issue, in part, on page 5 of Bulletin 2004-01 stating, ". . . the NRC staff believes that the topic of NDE scope expansion should be discussed with the NRC if circumferential PWSCC is observed in either the pressure boundary or non-pressure boundary portions of any locations covered under the scope of this bulletin to ensure that the licensee has performed an adequate extent-of-condition evaluation."

Because of the potential plant-specific and generic significance of circumferential PWSCC at locations covered under the scope of Bulletin 2004-01, it is the NRC staff's position that cognizant members of the Office of Nuclear Reactor Regulation's Materials and Chemical Engineering Branch (EMCB) should be promptly made aware of any emerging issue regarding this degradation phenomena at your facility. This is important not only for the reason cited in the passage above from Bulletin 2004-01, but also so that the NRC staff can evaluate any such information and fulfill its obligation to inform other U.S. nuclear power plant licensees of new operational experience that may be relevant to the continued safe operation of their facilities.

It is the NRC staff's expectation that if you obtain inspection results in the future that indicate that circumferential PWSCC may be occurring at any location covered under the scope of Bulletin 2004-01, you should contact your NRC Headquarters Project Manager (PM) and request a teleconference or meeting with EMCB technical staff. Notification of your NRC PM should allow ample time for you to incorporate any insights from the aforementioned teleconference or meeting into your plans for evaluating the extent of condition at your facility prior to the end of the outage during which the degradation was discovered.

In order to document your intent to follow the guidance provided above, please provide a supplement to your Bulletin 2004-01 item (1)(c) response that states:

If circumferential cracking is observed in either the pressure boundary or non-pressure boundary portions of any locations covered under the scope of this bulletin, [we] will develop plans to perform an adequate extent-of-condition evaluation and [we] will discuss those plans with cognizant NRC technical staff prior to restarting the affected unit.

Enclosure

Mr. James Scarola
Carolina Power & Light Company

Shearon Harris Nuclear Power Plant
Unit 1

cc:

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